

December 2, 2003

L-PI-03-100 10 CFR 2.202

U S Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
DOCKET NO. 50-306
LICENSE NO. DPR-60
60-DAY REPORT PURSUANT TO NRC ORDER EA-03-009 PARAGRAPH E FOR
2003 PRAIRIE ISLAND UNIT 2 REACTOR PRESSURE VESSEL HEAD INSPECTION

By letter dated February 11, 2003, the Nuclear Regulatory Commission issued Order EA-03-009. By letter dated March 3, 2003, Nuclear Management Company, LLC, (NMC) consented to the Order as written. Paragraph E of the Order requires a report detailing inspection results within 60-days after returning the plant to operation for each inspection required in Paragraph C of the Order and (if a leak or boron deposit was found during the inspection) for each inspection required in Paragraph D of the Order. In response to these requirements, NMC notes the following with respect to the reactor pressure vessel head inspection conducted during the past Prairie Island Unit 2 refueling outage:

Prairie Island Unit 2 was shutdown for refueling in September 2003 and the reactor pressure vessel (RPV) head was inspected as required by Paragraph C of the Order. The inspection of the bare metal head of the reactor vessel was completed on September 25, 2003. Inspection of 100% of the bare metal of the reactor head as well as 360 degrees around each penetration was achieved with the aid of mirrors. Acceptance criteria was: lack of any relevant indications of the type described in related Electric Power Research Institute (EPRI) guidance, namely evidence of any leakage arising from the penetration to head interface, and the lack of any boric acid accumulations on the carbon steel head surfaces that may result in corrosion. No evidence of such leakage was observed and no evidence of any boric acid accumulation was seen. Based on the lack of any relevant indications, the reactor head was determined to be acceptable for continued service.

In addition, visual inspections were performed (as required by Paragraph D of the Order) to identify potential boric acid leaks from pressure-retaining components above the RPV head. One relevant indication of boric acid leakage was

USNRC L-PI-03-100 Page 2

identified. Valve 2RC-8-33, Reactor Head Vent Orifice Bypass Valve, had evidence of boric acid leakage from the valve packing onto the pipe and insulation above the RPV head. The mirror insulation had a small dried puddle on it, but it was not near any seams and, thus, did not affect the RPV head. The boric acid was cleaned, the old packing was removed, the stuffing box was cleaned, and the valve was repacked. The valve was reinspected as part of the reactor coolant system integrity test and the head vent system inservice pressure test with no evidence of leakage.

In this letter we have made no new Nuclear Regulatory Commission commitments. Please contact Jeff Kivi (651-388-1121) if you have any questions related to this letter.

Úoseph M. Solymossy

Site Vice President,\Prairie Island Nuclear Generating Plant

CC Regional Administrator, USNRC, Region III
Project Manager, Prairie Island Nuclear Generating Plant, USNRC, NRR
NRC Resident Inspector – Prairie Island Nuclear Generating Plant